East Building, PHH-23
1200 New Jersey Ave, SE
Washington, D.C. 20590

U.S. Department of Transportation

Pipeline and Hazardous Materials Safety Administration

COMPETENT AUTHORITY CERTIFICATION FOR A TYPE B(M)F FISSILE RADIOACTIVE MATERIALS PACKAGE DESIGN CERTIFICATE USA/9975/B(M)F-96, REVISION 6

The Competent Authority of the United States certifies that the radioactive material package design described in this certificate satisfies the regulatory requirements for a Type B(M)F package for fissile material as prescribed in the regulations of the International Atomic Energy Agency[1] and the United States of America[2] The package design is approved for use within the United States for import and export shipments made in accordance with applicable international and domestic transport regulations.

1. Package Identification - 9975.

2. Package Description and Authorized Radioactive Contents - as described in U.S. Department of Energy Certificate of Compliance No. 9975, Revision 16 (attached). The package is designated as a Type B(M)F, since the maximum normal operating pressure is greater than 700kPa (100 psig).

3. Criticality - The minimum criticality safety index is 2.0. The maximum number of packages per conveyance is determined in accordance with Table 11 of the IAEA regulations cited in this certificate.

4. General Conditions -

   a. Each user of this certificate must have in his possession a copy of this certificate and all documents necessary to properly prepare the package for transportation. The user shall prepare the package for shipment in accordance with the documentation and applicable regulations.

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[1] "Regulations for the Safe Transport of Radioactive Material, 2012 Edition, No. SSR-6" published by the International Atomic Energy Agency (IAEA), Vienna, Austria.

[2] Title 49, Code of Federal Regulations, Parts 100-199, United States of America.

b. Each user of this certificate, other than the original petitioner, shall register his identity in writing to the Office of Engineering and Research, (PHH-23), Pipeline and Hazardous Materials Safety Administration, U.S. Department of Transportation, Washington D.C. 20590-0001.

c. This certificate does not relieve any consignor or carrier from compliance with any requirement of the Government of any country through or into which the package is to be transported.

d. This certificate provides no relief from the limitations for transportation of plutonium by air in the United States as cited in the regulations of the U.S. Nuclear Regulatory Commission 10 CFR 71.88.

e. Pursuant to Paragraph 558 of the IAEA Regulations, for each shipment made in accordance with this certificate, the shipper shall notify the competent authorities of each country through or into which the shipment is to be transported. The shipper shall ensure that this notification has been received by the competent authority at least 15 days prior to the commencement of the shipment.

f. Records of Management System activities required by Paragraph 306 of the IAEA regulations[1] shall be maintained and made available to the authorized officials for at least three years after the last shipment authorized by this certificate. Consignors in the United States exporting shipments under this certificate shall satisfy the applicable requirements of Subpart H of 10 CFR 71.

5. Special Conditions -

   a. Transport by air is not authorized.

6. Marking and Labeling - The package shall bear the marking USA/9975/B(M)F-96 in addition to other required markings and labeling.

7. Expiration Date - This certificate expires on August 31, 2028.

CERTIFICATE USA/9975/B(M)F-96, REVISION 6

This certificate is issued in accordance with paragraph(s) 813 and 816 of the IAEA Regulations and Section 173.471 and 173.472 of Title 49 of the Code of Federal Regulations, in response to the October 22, 2024 petition by Department of Energy, Washington, DC, and in consideration of other information on file in this Office.

Certified By:

for William Schoonover
Associate Administrator for Hazardous Materials Safety

December 19, 2024
(DATE)

Revision 6 - Issued to endorse U.S. Department of Energy Certificate of Compliance No. USA/9975/B(M)F-96(DOE), Revision 16 (attached).

U.S. DEPARTMENT OF ENERGY

DOE Packaging Certification Program

CERTIFICATE OF COMPLIANCE
For Radioactive Materials Package

DOE F 5822.1
(5-85 Formerly EV-618)

1a. Certificate Number	1b. Revision No.	1c. Package Identification No.	1d. Page No.	1e. Total No. Pages
9975	16	USA/9975/B(M)F-96 (DOE)	1	13

2. PREAMBLE

2a. This certificate is issued under the authority of 49 CFR Part 173.7(d).

2b. The packaging and contents described in item 5 below meet the safety standards set forth in subpart E, "Package Approval Standards" and subpart F, "Package, Special Form, and LSA-III Tests" Title 10, Code of Federal Regulations, Part 71.

2c. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.

3. This certificate is issued on the basis of a safety analysis report of the package design or application --

(1) Prepared by (*Name and Address*):	(2) Title and identification of report or application:	(3) Date:
U.S. Department of Energy Savannah River Operations Office P.O. Box A Aiken, South Carolina 29808	Safety Analysis Report for Packaging Model 9975, S-SARP-G-00003, Revision 5, March 2024, as supplemented [See 5(e)].	March 2024

4. CONDITIONS

This certificate is conditional upon the fulfilling of the applicable Operational and Quality Assurance requirements of 49CFR parts 100-199 and 10CFR Part 71, and the conditions specified in item 5 below.

5. Description of Packaging and Authorized Contents, Model Number, Transport Index, Other Conditions, and Supplements:

(a) Packaging

(1) Model: 9975

(2) Description:

The components of the packaging include the drum, insulation, bearing plates, primary containment vessel (PCV), secondary containment vessel (SCV), lead shielding, and aluminum honeycomb spacers. An aluminum PCV sleeve or 3013 top and bottom spacer may be used, depending on the type of product can to be transported. The nominal net weight of the packaging ranges from 350-374 lb. The drum is fabricated as a 35-gallon bolted lid drum of 18-gauge Type 304L stainless steel. Four ½-inch diameter vent holes are drilled into the drum, approximately 90 degrees apart, 1 inch below the drum flange and are covered with plastic Caplugs (fusible plugs). The plugging devices prevent water from entering the drum through the vent holes under normal conditions of transport. In the event a fire occurs, the plugs melt, allowing the drum to vent gases generated from the insulation to prevent rupture of the drum. The drum lid is bolted to a 1¼-inches wide by ⅛-inch thick angle flange welded to the top of the drum body using ½-inch high-strength bolts.

6a. Date of Issuance: July 22, 2024	6b. Expiration Date: August 31, 2028

FOR THE U.S. DEPARTMENT OF ENERGY

7a. Address *(of DOE Issuing Office)*	7b. Signature, Name, and Title *(of DOE Approving Official)*
U.S. Department of Energy Office of Packaging and Transportation (EM-4.24) 1000 Independence Avenue, SW Washington, DC 20585	Julia C. Shenk Digitally signed by Julia C. Shenk Date: 2024.07.22 17:08:12 -04'00' Julia C. Shenk Headquarters Certifying Official Director Office of Packaging and Transportation

The lid is recessed 0.55 inches. A ⅛-inch thick by 1¼- inches wide circular ring is welded to the outer section of the lid. The ring serves to reinforce the lid and prevents the lid from shearing away from the bolts during a hypothetical accident condition event. Nuts are tack welded to the flange underside to ease assembly operations. The bolts are tightened to 30 ± 2 ft-lb of torque.

The insulation material that surrounds the containment vessels is fiberboard/Celotex® (cane or softwood, Type IV Grade 1 per ASTM C-208-95, density 14-16 lb/ft^3), which is manufactured per ASTM Specification C-208-95. The fiberboard is regular grade wall sheathing material with a nominal density of 15 lb/ft^3 and comes in ½-inch thick sheets that are bonded together into top and bottom subassemblies with a water-based carpenter's glue. The insulation subassemblies are fitted to the drum so that the radial clearances between the insulation, the lead cylinder, and the drum do not exceed ¼ inch. The radial thickness of the insulation is 4¾ inches. In the axial direction, the top thickness of fiberboard/Celotex® is 3.5 inches and the bottom thickness is 3.4 inches. Placed over and glued to the top fiberboard/Celotex® subassembly is an air shield made of stainless steel.

A ½-inch thick Firemaster® encapsulated blanket is placed between the top insulation subassembly and the drum closure lid. The blanket is manufactured from a ceramic fiber (Kaowool®), encapsulated in stainless steel foil and heat-sealed.

Radiation shielding is provided by a shielding assembly (Drawing R-R2-F-0020) that surrounds the PCV/SCV double-containment assembly. The shielding assembly consists of a jacketed cylindrical shielding body subassembly (Drawing R-R2-F-0020-C) and aluminum lid (Drawing R-R2-F-0020-B). The lead of the shielding body is ASTM B-749 or B29 and machined, after casting, to a nominal thickness of ½-inch. The shielding body interior and exterior sides and base are covered in 20-gauge 304L stainless steel. The lid of the shielding body is ½-inch thick ASTM B-209 1100 aluminum and attaches to the shielding body with four screws. The combined thicknesses of stainless steel in the PCV and SCV closure provide equivalent shielding at the top of the shielding assembly in lieu of lead.

Two ½-inch thick aluminum bearing plates are added to the packaging to provide additional load-bearing surfaces against the fiberboard/Celotex® insulation.

The PCV consists of a stainless steel pressure vessel that is designed, analyzed, and fabricated in accordance with Section III, Subsection NB of the ASME Boiler and Pressure Vessel Code (B&PVC), 1992 edition, with a design condition of 900 psig at 300°F. The PCV is fabricated from 5-inch, Schedule 40, seamless, Type 304L stainless steel pipe (0.258-inch nominal wall) and has a standard Schedule 40, Type 304L stainless steel pipe cap (0.258-inch nominal wall) at the blind end. A 304L stainless steel cone seal flange is welded at the open end. Both vessel body joints are circumferential full-penetration butt welds examined by radiographic and liquid penetrant methods. These welds satisfy ASME B&PVC Section III, Subsection NB requirements.

A 4-inch, Schedule 40 pipe of the same material is welded to the convex side of the cap to form a skirt to vertically support the PCV.

The PCV closure consists of a male-female cone joint with surfaces that have been machined to identical angles so that they mate with zero clearance. Two grooves for O-rings have been machined into the face of the Type 304L stainless steel male cone. A leak test port is provided between the two O-ring grooves. A small (0.063-inch wide by 0.06-inch deep) rectangular groove is recessed into the face of the male cone between the two O-ring grooves, to ensure helium detection during leakage testing. Two Viton® GLT and/or Viton® GLT-S fluoroelastomer O-rings (greased with high-vacuum silicone grease) are placed in the grooves to form a leaktight seal (less

than 10^{-7} ref. cm^3/sec air). A Nitronic® 60 seal nut, which forces the male cone against the female cone, is threaded into the containment vessel body. The PCV has a gross internal volume of approximately 313 $inch^3$, weighs 34 lb., and is 18.6 inches long, with a usable inside cavity 15 inches deep with a minimum diameter of 5 inches. For certain oxide contents, the PCV (or PCV and SCV) is backfilled with an inert gas prior to closing.

An aluminum honeycomb spacer is inserted into the concave cavity of the PCV to provide a flat horizontal surface for the product cans. For some containment vessels, an additional bottom spacer is used.

The SCV consists of a stainless steel pressure vessel that is designed, analyzed, and fabricated in accordance with Section III, Subsection NB of the ASME B&PVC, 1992 edition, with design conditions of 800 psig at 300°F. The SCV is fabricated from 6-inch, Schedule 40, seamless, Type 304L stainless steel pipe (0.280-inch nominal wall) and has a standard Schedule 40, Type 304L stainless steel pipe cap (0.280-inch nominal wall) at the blind end. A 304L stainless steel cone seal flange is welded at the open end. Both vessel body joints are circumferential full-penetration butt welds examined by radiographic and liquid penetrant methods. These welds satisfy ASME B&PVC Section III, Subsection NB requirements.

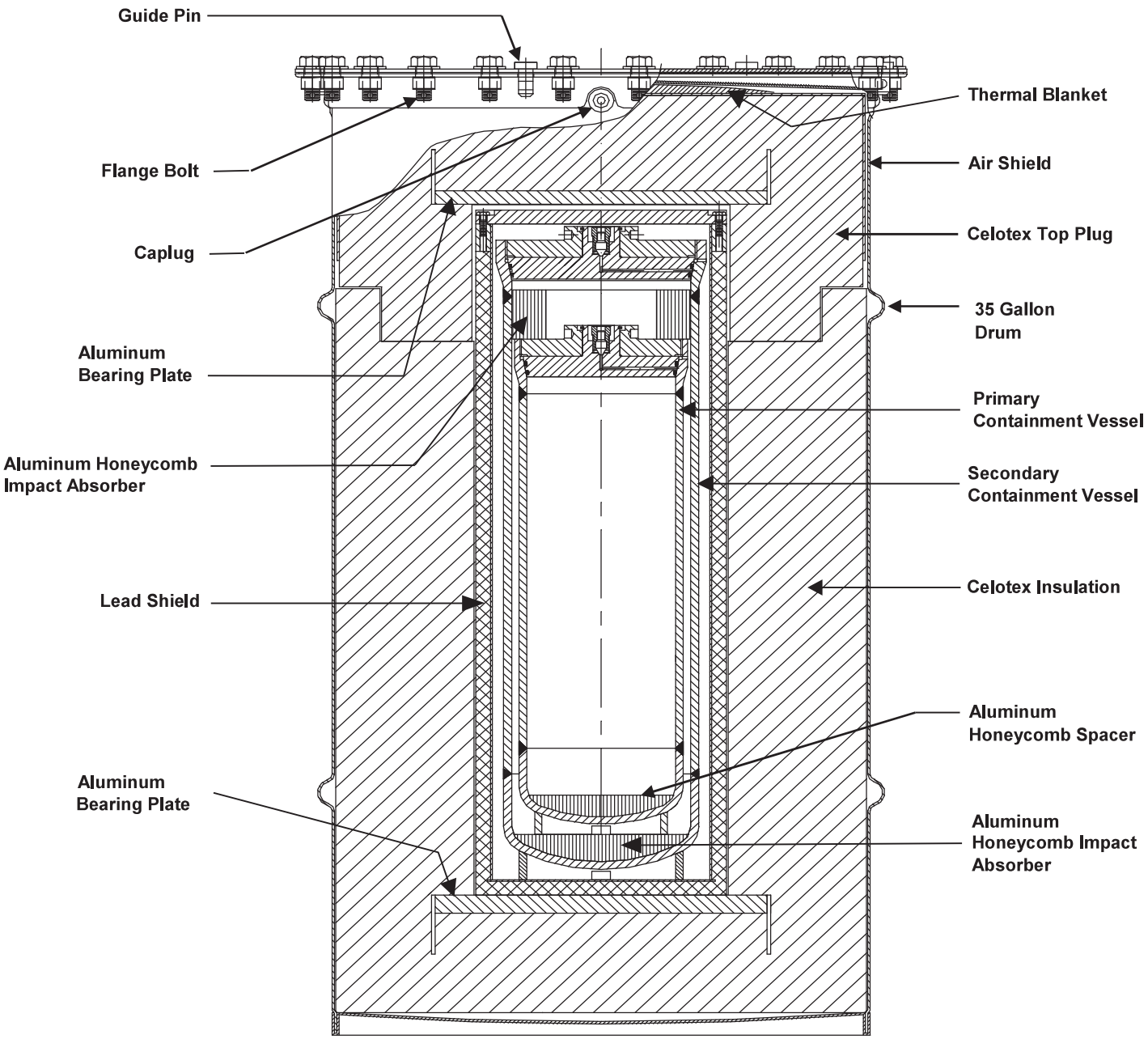
A 5-inch, Schedule 40 pipe of the same material is welded to the convex side of the cap to form a skirt to vertically support the SCV. The SCV closure is identical to that used on the PCV except that the SCV is 1 inch larger in diameter.

The SCV has a gross internal volume of approximately 604 $inch^3$, weighs 56 lb., and is 24 inches long. It has a usable cavity approximately 21.5 in. deep, with a minimum diameter of 6 in.

The aluminum honeycomb impact absorbers that fit axially between the PCV and SCV are fabricated from 3-mil minimum foil thickness. The impact absorbers are rated for an axial compressive strength before deformation of 1500 ± 500 psi. The top impact absorber has the shape of a ring. The bottom impact absorber is machined on the bottom face to roughly fit the contour of the inside of the SCV.

In some cases, for Content Envelope C.9, a small or large bore shielded-pig convenience container configuration is used for added shielding [see Drawings 5(a)(3)]. Both configurations consist of machined lead pig in an aluminum convenience can that is placed inside the PCV, between top and bottom aluminum honeycomb spacers, as shown in Figure 1-8 of the SARP.

Option to use the radiofrequency identification (RFID) system: The option to use the ARG-US RFID system is authorized per this certificate and the RFID Guide (SARP Section 1.3, Reference 19). ARG-US RFID tag is not considered a part of the package.



9975 PACKAGING

(3) Drawings:

The packaging design is defined by the following Savannah River Site drawings:

Drawing No.	Rev	Title/Notes
R-R2-F-0026	5	9975 Shipping Package Drum with Flange Closure Assembly (U)
R-R2-F-0019	8	9975 Shipping Package Insulation Assembly, Subassemblies and Details (U)
R-R2-F-0020	11	9975 Shipping Package Shielding (U). **Note -** Only R-R2-F-0020-C, 9975 Jacketed Steel Body Subassembly, is authorized; R-R2-F-0020-A, 9975 Steel Body Subassembly is prohibited.
R-R2-F-0025	7	9975 Drum with Flange Closure Subassembly and Details (U) Sheets 1 &2
R-R2-F-0018	10	9975 Shipping Package Primary and Secondary Containment Vessel Subassemblies (U)
R-R3-F-0016	13	9975 Shipping Package Containment Vessel Weldments (U)
R-R3-F-0015	6	9975 Shipping Package Air Shield Weldment (U)
R-R4-F-0054	14	9975 Shipping Package Primary (PCV) & Secondary (SCV) Containment Vessel Details (U)
R-R4-F-0055	5	9975 Shipping Package PCV Sleeve and 3013 Top Spacer Details (U)
R-R2-F-0037	1	9975 Packaging Alternate 3013 Spacer Components Details (U)
R-R4-G-00047	1	U-233 Lead Pig Details and Subassembly (U)
R-R4-G-00048	1	U-233 Container Details and Subassembly (U)
R-R4-G-00051	3	U-233 Spacers (U)
R-R4-G-00166	0	U-233 Large Bore Lead Pig

(b) Contents:

Radioactive material must be packaged in internal containers of varying designs/configurations per Section 1.2.3 of the SARP.

(1) Type and Form of Material: (See Table 1)

(i) Uranium metal or oxide as specified in Content Envelope C.1.

(ii) Plutonium-238 heat sources as specified in Content Envelope C.2.

(iii) Plutonium and/or uranium metal as specified in Content Envelope C.3 or C.10.

(iv) Plutonium and/or uranium oxide as specified in Content Envelope C.4 or C.11.

(v) Plutonium composites as specified in Content Envelope C.5.

(vi) Plutonium/tantalum composites as specified in Content Envelope C.6.

(vii) Plutonium-238 oxide/beryllium metal as specified in Content Envelope C.7.

(viii) Neptunium oxide as specified in Content Envelope C.8.

(ix) Uranium 233 oxides and metals as specified in Content Envelope C.9.

Table 1 - Content Envelopes

	Material [a, b]	C.1 [c, d]	C.2	C.3 [d, e, f, g]	C.4 [e, g, h, i]	C.5 [d, f, g, j]	C.6 [d, f, k]	C.7	C.8 [l]
		U Metal/Oxide	^{238}Pu Heat Sources	Pu/U Metals	Pu/U Oxides	Pu Composites	Pu/Ta Composites	^{238}Pu Oxide/ Be Metal	Neptunium Oxide
Radioisotope [m] (Weight Percent of Radioactive Material Mass)	^{236}Pu	-	1×10^{-4}	-	-	-	-	1×10^{-4}	-
	^{238}Pu [n]	-	100	2	2	0.05	0.05	80	5×10^{-2}
	^{239}Pu [o]	-	40	100	100	100	100	40	gg
	^{240}Pu	-	13	50	50	6.5	6.5	13	gg
	^{241}Pu [o, p]	-	1	15	15	1	1	1	gg
	^{242}Pu	-	1.5	5	5	0.1	0.1	1.5	gg
	^{241}Am + ^{241}Pu	-	1	15	15	1	1	1	hh
	^{243}Am	-	1×10^{-4}	1×10^{-4}	1×10^{-4}	1×10^{-4}	1×10^{-4}	1×10^{-4}	
	^{244}Cm	-	1×10^{-4}	1×10^{-4}	1×10^{-4}	1×10^{-4}	1×10^{-4}	1×10^{-4}	
	^{237}Np	-	0.5	5.0	5.0	-	-	-	100
	^{232}U	1×10^{-5}	4×10^{-6}	1×10^{-5}	1×10^{-5}	-	-	-	
	^{233}U [o]	0.5	0.2	0.5	0.5	-	-	-	ii
	^{234}U [q]	100	40	100	100	-	-	-	ii
	^{235}U [o]	100	40	100	100	-	-	-	ii
	^{236}U	40	16	40	40	-	-	-	ii
	^{238}U	100	40	100	100	-	-	-	ii
	^{232}Th	-	10	23 [x]	23 [x]	-	-	-	ii
Impurities [kk] (grams)	Al, B, F, Li, Mg, Na	-	-	[r]	[r]	-	-	-	jj
	Be	-	-	500	500	4,400	-	200 [s]	jj
	V	-	-	-	-	4,400	-	-	jj
	Ta	-	-	-	-	4,400	6,000	-	jj
	C	-	-	1,000	1,000	-	-	-	jj
Total Mass (kilograms)	Radioactive Materials	13.5	0.1	4.4	4.4	4.4	2	0.02	6
	Impurities	-	-	3.08 [t]	3.08 [t]	4.4	6 [u, v]	0.2	0.15
	All Contents	13.5	0.1	4.4	5	4.4	8	0.22	6.81

Table 1 - Content Envelope (Continued)

	Material [a, b]	C.9 ^{233}U Metal/ Oxides grams	C.10 [d, e, f, g] P/U Metals grams	C.11 [e, g, h, i] Pu/U Oxides grams
Radioisotope [m] (Radioactive Material Mass Grams)	^{236}Pu [z]	-	-	-
	^{238}Pu [n]	-	34	34
	^{239}Pu [o]	aa	4400	4400
	^{240}Pu	bb	1450 [y]	2200
	^{241}Pu [o, p]	bb	188.9	188.9
	^{242}Pu	-	400 [y]	2200
	^{241}Am + ^{241}Pu	bb	188.9	188.9
	^{243}Am	-	1.00	1.00
	^{244}Cm	-	0.0044	0.0044
	^{237}Np	-	220	220
	^{232}U	0.0018 [cc]	0.00044	0.00044
	^{233}U [o]	500 [dd]	427	427
	^{234}U [q]	aa	4400	4400
	^{235}U [o]	aa	4400	4400
	^{236}U	4400	2640	2640
	^{238}U	4400	4400	4400
	^{232}Th	-	4400	4400
Impurities [ee, kk] (grams)	Al, B, F, Li, Mg, Na	-	r	r
	Be	-	500	500
	V	-	-	-
	Ta	-	-	-
	C	-	1000	1000
Total Mass (kilograms)	Radioactive Materials	4.4	4.4	4.4
	Impurities	-	3.08 [t]	3.08 [t]
	All Contents	4.4	4.4	5

Table 1 – Table Notes

a	Except as permitted for oxides, all contents shall be dry.
b	Pu/U content bulk density shall be no greater than 19.84 g/cc. No minimum bulk density is specified. However, low bulk densities require dilution of the local atmosphere within the content container by a specific gas (helium or nitrogen) and/or reduction in the allowable decay heat as summarized in Table 3.4 of SARP.
c	Up to 1 gram of plutonium contamination is permitted.
d	Each unclad metal piece shall have a minimum thickness of 1.0 mm (0.04 inches) and a specific surface area less than 100 mm^2/gram (71 in^2/lb) per DOE-STD-3013-2004, as superseded. A minimum 50-gram mass per metal piece conservatively meets these criteria.
e	Mass limit due to shielding. The heat loading of each mixture needs to be determined. The 188.9 gram limit based on estimate of heat load from WCID-2009-0002 Revision 0.
f	Contents shall be stabilized in accordance with DOE-STD-3013-2004, as superseded, Section 6.1. Engineered metal materials with intact cladding meet the requirements for stabilized metals under content envelopes C.3 and C.10
g	Plutonium plus uranium mass shall not be less than 30 weight percent of the total content mass.
h	Contents shall be stabilized in accordance with DOE-STD-3013-2004, as superseded, Section 6.1. Engineered oxide materials with intact cladding meet the requirements for stabilized oxides under content envelopes C.4 and C.11
i	The moisture content of the oxide shall be less than 0.5 weight percent of the total content mass.
j	Depleted Uranium or Enriched Uranium may be substituted for any amount of plutonium on a gram-for gram basis.
k	The Be, V, and Ta each form a composite with the radionuclide content and, as such, are not homogeneously mixed with the Pu. A maximum of 50 pieces of composite material is permitted.
l	Material to be prepared in accordance with WSRC-TR-2003-00388 which limits the moisture content of the material.
m	Maximum amounts by constituent.
n	^{238}Pu decays to ^{234}U, which will result in significant concentrations of ^{234}U over time. ^{234}U growth will not adversely impact package performance.
o	Nuclide classified as “fissile” per DOE Good Practices Guide, Criticality Safety Good Practices Program, Guide For DOE Nonreactor Nuclear Facilities, DOE G 421.1-1, 3.79 *Fissile Nuclide*, 8-25-99.
p	^{241}Pu must be less than ^{240}Pu.
q	Applies to ^{234}U other than ^{234}U resulting from ^{238}Pu decay.
r	The listed light element impurities have a combined mass limit of 3080 grams minus the mass of Be and C present.
s	The beryllium is assumed to be physically separated from the plutonium oxide. The 200 grams of beryllium can be in any configuration with up to 275 cm^2 in direct contact with plutonium contents. The surface area restriction is based on shielding.
t	Total impurity limit is based on the minimum 30% Pu + U mass within DOE-STD-3013-2004, as superseded. The limit was calculated from the maximum radioactive material mass (4.4 kg). [4.4 kg × 70% = 3.08 kg]

Table 1 – Table Notes (continued)

u	Plutonium mass is assumed bonded to the tantalum (as an outer/inner reflector) and is not readily separable.
v	For analytical purposes there are no mixing assumptions for the Ta with the radionuclide content.
w	Reserved
x	Not to exceed 1000 grams total ^{232}Th.
y	The ^{240}Pu and the ^{242}Pu mass limits may be adjusted per the equation: ^{242}Pu + 0.596 ^{240}Pu < 1290, Where ^{240}Pu and ^{242}Pu are the mass limits in grams.
z	^{236}Pu is not expected to be present in significant amounts
aa	These isotopes may be present as long as their contribution as equivalent ^{233}U in the package combined with the actual ^{233}U content present does not exceed the ^{233}U mass content limit. The "equivalent ^{233}U" mass is given by the equation ^{233}U (eq) = ^{233}U + ^{235}U/1.4 + ^{239}Pu/0.83 for the three primary fissile isotopes where each isotope name indicates the mass of that isotope. Any ^{234}U present shall be considered ^{233}U for this equation.
bb	Small quantities (< 1 g) of these isotopes may be present as long as the ^{240}Pu mass exceeds the ^{241}Pu mass, their combined mass is less than the ^{239}Pu mass, and these isotopes are treated as ^{239}Pu mass in the determination of ^{233}U (eq) mass.
cc	0.0018 grams is the limiting mass of ^{232}U based on the 9975 package shielding. The ^{232}U mass limit increases to 0.004 grams if the large bore shielded-pig (Drawing R-R4-G-00166) is used or 0.0101 grams if the small bore shielded-pig (Drawing R-R4-G-00047) is used.
dd	This mass value is the minimum subcritical mass limit for ^{233}U (ANSI/ANS-8.1).
ee	When present, nickel is plating used to fix contamination on the welded stainless steel capsule encasing the uranium source material for content envelope C.9
ff	^{237}Np must comprise at least 97.5 % weight percent of the Radioactive Material Mass
gg	The listed plutonium isotopes have a combined mass limit of 6.11×10^{-2} wt.% of the Radioactive Material Mass minus the mass of ^{238}Pu present
hh	Random material sampling for ^{241}Am has shown that the levels are "Less than Detectable". Since ^{241}Am is only a shielding concern, conformance to 10 FR 71 dose rate limits shall be demonstrated through the Dose Rate Measurement Methodology described in the 9975 SARP. Therefore the measurement of ^{241}Am is not required. The ^{241}Pu limit in Table note gg applies.
ii	The total amount of all radioisotopes other than neptunium must be less than 2.5 wt.%. For plutonium isotopes, the more restrictive limits of Table note gg apply.
jj	The total non-radioactive impurities must be less than 2.2 wt.% of the neptunium oxide mass.
kk	Gallium may be present in stabilized oxide and metal plutonium contents up to the combined 3013 impurity limit of 3080 grams as described in footnote t unless a lower impurity limit applies to the content envelope.

(2) Maximum Quantity of Material per Package: as specified in Tables 1, 2 or 3.

(i) For all Content Envelopes:

(a) The maximum decay heat per package may not exceed 19 watts. Content envelopes C.4 and C.11 are shown in Table 2 below for the decay heat limits.

Table 2. Requirements for Local Atmosphere Dilution and Decay Heat

C.4 &C.11 Density (g/cm^3)	Local Atmospheres Inside PCV		Maximum Decay Heat (Watts)
	Inside Content Container	Outside Content Container	
2.0 to 19.84	N_2 or Helium dilution to: ≤ 5% O_2	≥ 75% CO_2 ≤ 25% air	19
1.0 to < 2.0	Helium dilution to: ≤ 5% O_2	↓	19
1.0 to < 2.0	N_2 dilution to: ≤ 5% O_2	↓	18
< 1.0	Helium dilution to: ≤ 5% O_2	↓	18.4
< 1.0	N_2 dilution to: ≤ 5% O_2	↓	16.5

The maximum weight of all material (radioactive contents, product cans, spacer, shielded pig, etc.) inside the PCV may not exceed 20.1 kg (44.4 lb.).

(c) Except as permitted for oxides, all contents shall be dry.

(d) Pu/U content bulk density shall be no greater than 19.84 g/cc. No minimum bulk density is specified. However, low bulk densities may require dilution of the local atmosphere within the content container by a specific gas (helium or nitrogen) and/or reduction in the allowable decay heat as summarized in Table 3.4 of the SARP. Not applicable to Content Envelope C.9.

(e) Except as stated in Table 1, small concentrations (<1000 ppm each) of other actinides, fission products, decay products, and neutron activation products are permitted.

(ii) For Content Envelope C.1:

(a) Up to 1 gram of plutonium contamination is permitted.

(b) Each metal piece shall have a minimum thickness of 1.0 mm (0.04 inches) and a specific surface area less than 100 mm^2/g (71 $in^2/lb.$) per DOE-STD-3013-2004, as superseded.

(iii) For Content Envelopes C.3 or C.10:

(a) Each unclad metal piece shall have a minimum thickness of 1.0 mm (0.04 inches) and a specific surface area less than 100 mm^2/g (71 $in^2/lb.$) per DOE-STD-3013-2004, as superseded.

(b) Contents shall be stabilized in accordance with DOE-STD-3013-2004, as superseded, Section 6.1. Engineered metal materials with intact cladding meet the requirements for stabilized metals under content envelopes C.3 and C.10.

(c) Plutonium plus uranium mass may not be less than 30 weight percent of the total content mass.

(iv) For Content Envelopes C.4 or C.11:

(a) Plutonium plus uranium mass may not be less than 30 weight percent of the total content mass.

(b) Contents shall be stabilized in accordance with DOE-STD-3013-2004, as superseded, Section 6.1. Engineered oxide materials with intact cladding meet the requirements for stabilized oxides under content envelopes C.4 and C.11.

(c) The moisture content of the oxide shall be less than 0.5 weight percent of the total content mass.

(d) The PCV sleeve is not required for food-pa*ck* or engineered convenience can configurations when the outer can diameter exceeds 4.38 inches and the PCV shall be inerted with nitrogen so that at the time of closure the oxygen content in all void spaces is no greater than 5% by volume [See SARP Section 1.2.3.2.2]. For oxide contents under this configuration without the PCV sleeve the total mass of empty food-pack or engineered convenience cans must be less than 9,000 grams.

(v) For Content Envelope C.5:

(a) Each metal piece shall have a minimum thickness of 1.0 mm (0.04 inches) and a specific surface area less than 100 mm^2/g (71 $in^2/lb.$) per DOE-STD-3013-2004, as superseded.

(b) Contents shall be stabilized in accordance with DOE-STD-3013-2004, as superseded, Section 6.1.1.

(c) Plutonium plus uranium mass may not be less than 30 weight percent of the total content mass.

(d) Depleted uranium or enriched uranium may be substituted for any amount of plutonium on a gram-for-gram basis.

(vi) For Content Envelope C.6:

(a) Each metal piece shall have a minimum thickness of 1.0 mm (0.04 inches) and a specific surface area less than 100 mm^2/g (71 $in^2/lb.$) per DOE-STD-3013-2004, as superseded.

(b) Contents shall be stabilized in accordance with DOE-STD-3013-2004, as superseded, Section 6.1.1.

(c) A maximum of 50 pieces of composite material is permitted.

(vii) For Content Envelope C.7:

(a) The 200 grams of beryllium can be in any configuration with up to 275 cm^2 in direct contact with plutonium contents.

(viii) For Content Envelope C.8:

(a) Material shall be prepared in accordance with WSRC-TR-2003-00388, which limits the moisture content of the material.

(b) A total of 100 grams of plastic may be present as low-density polyethylene bags or nylon bagging and polyvinyl chloride tape.

(c) Use of the PCV sleeve or aluminum foil packing material is prohibited.

(d) Metal mass of food-pack or engineered convenience cans is restricted to 1,000 grams.

(e) ^{240}Pu content must be greater than the ^{241}Pu content.

(f) The measured Loss on Ignition (LOI) of the product must be less than 0.24 wt.%.

(g) Neptunium Oxide content must have a measured assay of at least 86.2 wt.% neptunium, which corresponds to at least 97.5% neptunium of the total radioactive material.

(h) The total plutonium content must not exceed 611 micrograms per gram of neptunium.

(i) The total plutonium α (alpha) activity must not exceed 8,580 microcuries per gram of neptunium.

(j) The total non-radioactive impurities must be less than 2.2 wt.% of the oxide mass.

(j) All containers (food-pack or engineered convenience cans, PCV, and SCV) shall be inerted with argon, such that oxygen content in all void spaces is no greater than 3% by volume at closure.

(ix) For Content Envelope C.9:

(a) PCV bottom spacer is required.

(b) Shipments are not authorized in a 3013 or Hex-can

(c) If the mass of U-232 does not exceed 0.0018 grams, contents may be packaged in a food-pack or engineered convenience can, with a maximum of 100 g plastic. Aluminum pellets or foil for packaging is allowed. Oxide contents must be calcined to at least 600°C and encapsulated in stainless steel or nickel alloy.

(d) If the mass of U-232 exceeds 0.0018 grams the small or large bore shielded-pig configurations may be used subject to the following: the large bore shielded-pig configuration may be used if the U-232 mass does not exceed 0.004 grams or the small bore shielded-pig configuration may be used if the U-232 mass does not exceed 0.0101 grams.

Table. 3 – Shipping Period Based on Nitrogen Inerting, Moisture Level in MOX, and Initial Oxygen Concentration

Moisture (wt.%)	Shipping Period (Days)	
	1% initial O_2 (10,000 ppm)	2% initial O_2 (20,000 ppm)
2.42	18	12
2.0	23	15
1.5	32	20
1.0	51	33
.54	115	74
.50	129	83

(c) Criticality Safety Index: 2.0

(d) Conditions:

(1) Content envelope loading arrangements/configurations shall comply with the applicable requirements of Sections 1.2.3.1 and 1.2.3.2 of the SARP.

(2) Food-pack cans with organic liners may not be used for any contents.

(3) Internal content containers must be examined for post-sealing bulging or buckling prior to placement inside the PCV. No internal content container that has visibly bulged or buckled may be transported in the package.

(4) Internal content containers must be examined upon removal from the PCV after shipment. Any visible bulging, buckling, or evidence of corrosion on the exterior shall be reported to the Design

Agency (DA). The DA shall report to the DOE Headquarters Certifying Official any condition the DA deems significant to safety.

(5) The gross weight of the package may not exceed 404 lb.

(6) For the contents described in 5(b)(1)(v), any package that is subjected to an impact greater than that of a four-foot drop shall be surveyed for neutron dose rate prior to contact or handling.

(7) In addition to the requirements of Subparts G and H of 10 CFR Part 71, and except as specified in section 5(d) of this certificate, each package must be fabricated, acceptance tested, operated, and maintained in accordance with the Operating Procedures requirements of Chapter 7, Acceptance Tests and Maintenance Program requirements of Chapter 8, and packaging-specific Quality Assurance requirements of Chapter 9 of the SARP.

(8) Transport by air of fissile material is not authorized.

(9) Revision 15 of this certificate may be used until May 31, 2025, for DOE domestic transport of 9975-96 packages.

(10) Only DOE elements or persons working under contract to DOE elements shall consign the package for domestic transport.

(11) Nuclear Regulatory Commission (NRC) or Agreement State licensees shall not consign a DOE certified package for domestic transport but can transfer the material on-site to DOE elements or persons working under contract to DOE elements for consignment of the package.

(e) Supplements:

None

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U.S. Department of Transportation

Pipeline and Hazardous Materials Safety Administration

CERTIFICATE NUMBER: USA/9975/B(M)F-96

ORIGINAL REGISTRANT(S):

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