NRC FORM 618 U.S. NUCLEAR REGULATORY COMMISSION (8-2000) 10 CFR 71 CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES b. REVISION NUMBER a. CERTIFICATE NUMBER c. DOCKET NUMBER d. PACKAGE IDENTIFICATION NUMBER PAGE **PAGES** 71-9319 USA/9319/B(U)F-96 OF 9319 14 1 8

2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.
- 3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION
- a. ISSUED TO (Name and Address)
 Framatome Inc.
 2101 Horn Rapids Road
 Richland, WA 99354

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION Framatome Inc. application dated March 31, 2021.

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

CLEAR

5.

(a) Packaging

(1) Model Nos.: MAP-12 and MAP-13

(2) Description

The MAP package is designed to transport unirradiated uranium fuel assemblies with enrichment up to 8.0 weight percent. The package is designed to carry two fuel assemblies with core components. The package consists of two components: a base and lid. The containment system of the MAP package is the fuel rod cladding.

The base consists of a fixed stainless steel strong-back which supports the fuel assemblies. A series of inner stiffeners are secured to the underside of the strong-back to support the fuel assemblies. A neutron moderator and absorber are positioned directly beneath the strong-back between each inner stiffener. The base inner stiffeners are retained by a stainless steel cover. Exterior to the cover is a layer of rigid polyurethane foam and a stainless steel outer shell. A second stainless steel sheet is provided between the two middle stiffeners. Four stainless steel outer stiffeners support the package base. The payload rests on the "W" shaped strong-back (referred to as a W-plate) and is held in place with hinged and latched aluminum doors. The lid is very similar to that of the base – a "W" shaped stainless steel inner shell is fitted with a series of inner stiffeners, neutron moderator and absorbers, and a stainless steel cover is fitted over the stiffeners. The lid is fitted with trapezoidal impact limiters at each end. The impact limiters are constructed from rigid polyurethane foam encased by the package outer stainless shell skin. The base and lid include end plates with interlocking, interfacing angles.

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5.(a)(2) Description (continued)

There are two models of the MAP package, the MAP-12 and MAP-13. The weights and dimensions of the package are as follows:

MAP-12 (for 144-in Maximum Nominal Active Fuel Length):

Maximum Gross Weight 8,630 lbs Maximum Payload Weight 3,400 lbs

Outer Dimensions

Length 208 in Width 45 in Height 31 in

MAP-13 (for 150-in Maximum Nominal Active Fuel Length)

Maximum Gross Weight 8,930 lbs Maximum Payload Weight 3,400 lbs

Outer Dimensions

Length 221 in Width 45 in Height 31 in

(3) Drawings

The MAP-12 and MAP-13 packages are fabricated and assembled in accordance with the following Framatome Inc. Drawing Nos.:

9045393, Rev. 9;	9045402, Rev. 6;
9045397, Rev. 3;	9045403, Rev. 6;
9045399, Rev. 4;	9045404, Rev. 5;
9045401, Rev. 6;	9045405, Rev. 6.

(b) Contents

(1) Type and Form of Material

(i) ≤ 5.0 weight percent U-235

Enriched commercial grade uranium or slightly contaminated uranium with trace quantities limits as defined in Table 1 below. Uranium oxide fuel rods, enriched to no more than 5.0 weight percent in the U-235 isotope, with limits specified in Table 1 below.

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(ii) > 5.0 to ≤ 8.0 weight percent U-235

Enriched commercial grade uranium or slightly contaminated uranium with trace quantities limits as defined in Table 2 below.

Uranium oxide fuel rods in the 17x17 Type 3 array, enriched to no more than 8.0 weight percent in the U-235 isotope, with limits specified in Table 2 below.

(2) Maximum Quantity of Material per Package

Table 1: Maximum Authorized Concentrations for ≤ 5.0 weight percent U-235

Isotope	Maximum Content			
U-232	2.00 x 10 ⁻⁹ g/g U			
U-234 2.00 x 10 ⁻³ g/g l				
U-235 5.00 x 10 ⁻² g/g U				
U-236	2.50 x 10 ⁻² g/g U			
U-238	Balance of Uranium			
Np-237	1.66 x 10 ⁻⁶ g/g U			
Pu-238	6.20 x 10 ⁻¹¹ g/g U			
Pu-239	3.04 x 10 ⁻⁹ g/g U			
Pu-240	3.04 x 10 ⁻⁹ g/g U			
Gamma Emitters	6.38 x 10 ⁵ MeV – Bq/kg U			

Maximum content of U-238 is 9.23×10^{-1} g/gU for a maximum U-235 concentration of 5%. Since, for concentrations less than 5%, the U-238 value will be higher, it is shown as "Balance of Uranium" in Table 1.

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Table 2: Maximum Authorized Concentrations for > 5.0 to ≤ 8.0 weight percent U-235

Isotope	Maximum Content
U-232	1.10 x 10 ⁻⁷ g/g U
U-234	1.30 x 10 ⁻² g/g U
U-235	8.00 x 10 ⁻² g/g U
U-236	2.50 x 10 ⁻² g/g U
U-238	Balance of Uranium
Np-237	1.66 x 10 ⁻⁶ g/g U
Pu-238	6.20 x 10 ⁻¹¹ g/g U
Pu-239	3.04 x 10 ⁻⁹ g/g U
Pu-240	3.04 x 10 ⁻⁹ g/g U
Gamma Emitters	6.38 x 10 ⁵ MeV – Bq/kg U

Maximum content of U-238 is 8.82 x 10⁻¹ g/gU for a maximum U-235 concentration of 8%. Since, for concentrations less than 8%, the U-238 value will be higher, it is shown as "Balance of Uranium" in Table 2.

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(3) Fuel Assembly

(i) The parameters of the authorized fuel assemblies for ≤ 5.0 weight percent U-235 are specified in Table 3 below.

Table 3: Fuel Assembly Parameters for ≤ 5.0 weight percent U-235

Fuel Rod Array	142	x14	15x15					16x16	172	x17
Assembly Type	1	2		1		2	3	1	1	2
No. of Fuel Rods	176	179		208		216	204	236	264	264
No. of Non-Fuel Cells	20	17		17		9	21	20	25	25
Nominal Fuel Rod Pitch (in)	0.580	0.556		0.568		0.550	0.563	0.506	0.502	0.496
Maximum Pellet Outer Diameter (in)	0.3812	0.3682	0.3622	0.3707	0.3742	0.3617	0.3682	0.3282	0.3252	0.3232
Minimum Fuel Rod Outer Diameter (in)	0.438	0.422	0.414	0.428	0.428	0.414	0.422	0.380	0.377	0.372
Minimum Clad Wall Thickness ^(a) (in)	0.0245	0.0230	0.0220	0.0245	0.0230	0.0220	0.0230	0.0220	0.0220	0.0205
Minimum Guide Tube Wall Thickness (in)	N/A	N/A	0.0140	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Minimum Guide Tube Outer Diameter (in)	N/A	N/A	0.528	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Number of Guide Tubes	N/A	N/A	16	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Minimum Instrument Tube Wall Thickness (in)	N/A	N/A	0.0240	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Minimum Instrument Tube Outer Diameter (in)	N/A	N/A	0.491	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Number of Instrument Tubes	N/A	N/A	*	N/A	N/A	N/A	N/A	N/A	N/A	N/A
Clad/Tube Material Type ^(a)	Zr Alloy	Zr Alloy		Zr Alloy	Rest.	Zr Alloy	Zr Alloy	Zr Alloy	Zr Alloy	Zr Alloy
Maximum Active Fuel Length (in)	160	160		160		160	160	160	160	160

⁽a): the clad tube/material may include chromium coated cladding. The thickness of the cladding (maximum 20 µm) is not included as part of the minimum clad wall thickness.

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(ii) The parameters of the authorized fuel assemblies for ≤ 8.0 weight percent U-235 are specified in Table 4 below.

Table 4: Fuel Assembly Parameters for ≤ 8.0 weight percent U-235

Fuel Rod Array	17x17
Assembly Type	3
No. of Fuel Rods	264
No. of Non-Fuel Cells	25
Nominal Fuel Rod Pitch (in)	0.496
Maximum Pellet Outer Diameter (in)	0.3254
Minimum Fuel Rod Outer Diameter (in)	0.372
Minimum Clad Wall Thickness ^(a) (in)	0.0205
Minimum Guide Tube Wall Thickness (in)	N/A
Minimum Guide Tube Outer Diameter (in)	N/A
Number of Guide Tubes	N/A
Minimum Instrument Tube Wall Thickness (in)	N/A
Minimum Instrument Tube Outer Diameter (in)	N/A
Number of Instrument Tubes	N/A
Clad/Tube Material Type ^(a)	Zr Alloy
Gadolinia Requirements # @ wt% Gd ₂ O ₃ Lattice Average Enrichment ^(b)	
≤8.0 wt% U-235	12 @ 2 wt.%
≤7.0 wt% U-235	10 @ 2 wt.%
≤6.5 wt% U-235	8 @ 2 wt.%
≤6.0 wt% U-235	4 @ 2 wt.%
≤5.0 wt% U-235	No Gadolinia requirements
Maximum Active Fuel Length (in)	150

⁽a): The clad tube/material may include chromium coated cladding. The thickness of the cladding (maximum 20 μ m) is not included as part of the minimum clad wall thickness.

⁽b): Required gadolinia rods must be distributed symmetrically along the major diagonal.

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- Fuel Assembly (continued)
 - (ii) Non-fissile base-plate mounted and spider body core components are permitted.
 - (iii) Fuel rods assembled into the fuel assemblies are those loaded with sintered pellets of uranium oxides and/or with sintered pellets of uranium oxides mixed with various additives (e.g., Chromium, Boron, Gadolinium, and Europium).
- Criticality Safety Index for contents described (c) in Table 1 and Table 3 for ≤ 5.0 weight percent U-235:

Criticality Safety Index for contents described in Table 2 for ≤ 7.0 weight percent U-235:

J-235. Criticality Safety Index for contents described in Table 4 for ≤ 8.0 weight percent U-235:

- 6. In addition to the requirements of Subpart G of 10 CFR Part 71:
 - The package shall be prepared for shipment and operated in accordance with the Package (a) Operations in Section 7 of the application, as supplemented.
 - Each package must meet the Acceptance Tests and Maintenance Program of Section 8 of the (b) application, as supplemented.
 - Each fuel assembly must be unsheathed or must be enclosed in an unsealed, polyethylene or (c) polypropylene sheath, which may not extend beyond the ends of the fuel assembly. The ends of the sheath may not be folded or taped in any manner that would prevent the flow of liquids into or out of the sheathed fuel assembly.
 - The fuel rods must be leak tested after fabrication to ensure that the leakage rate of the (d) containment boundary is less than 10⁻⁷ ref cc/sec.
- 7. Transport by air of fissile material is not authorized.
- 8. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17.
- 9. Expiration date: February 29, 2028.

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REFERENCES

Framatome Inc., application "MAP PWR Fuel Shipping Package, FS1-0038397, Revision No. 5, dated March 31, 2021.

Renewal Request letter dated November 30, 2022.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Signed by Diaz-Sanabria, Yoira on 02/06/23

Yoira K. Diaz-Sanabria, Chief Storage and Transportation Licensing Branch Division of Fuel Management Office of Nuclear Material Safety and Safeguards

Date: See digital signature



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION REPORT
Docket No. 71-9319
Model Nos. MAP-12 and MAP-13
Certificate of Compliance No. 9319
Revision No. 14

SUMMARY

By letter dated November 30, 2022, Agencywide Documents Access and Management System Accession No. ML22334A253, Framatome Inc., (or the applicant) requested the renewal of its Certificate of Compliance (CoC) No. 9319 for the MAP packaging which had a CoC expiration date of February 28, 2023.

The certificate has been updated to Revision No. 14 and renewed for five years.

EVALUATION

The applicant did not request any changes in the package design, operating procedures, acceptance tests and maintenance program. No changes of contents of the package were also requested.

The staff reviewed the documents referenced in the CoC and determined that the package application was complete.

Condition No. 9 has been deleted and Condition No. 10, now renumbered Condition No. 9, has been revised to extend the CoC expiration date to February 29, 2028.

CONCLUSION

Based on the statements and representations in the application, the staff finds that this change does not affect the ability of the package to meet the requirements of Title 10 of the *Code of Federal Regulations*, Part 71.

Issued with CoC No. 9319, Revision No. 14.