NRC FORM 618 U.S. NUCLEAR REGULATORY COMMISSION (8-2000) 10 CFR 71 CERTIFICATE OF COMPLIANCE FOR RADIOACTIVE MATERIAL PACKAGES a. CERTIFICATE NUMBER b. REVISION NUMBER d. PACKAGE IDENTIFICATION NUMBER c. DOCKET NUMBER PAGE **PAGES** 9269 14 71-9269 USA/9269/B(U)-96 1 OF 3

2. PREAMBLE

- a. This certificate is issued to certify that the package (packaging and contents) described in Item 5 below meets the applicable safety standards set forth in Title 10, Code of Federal Regulations, Part 71, "Packaging and Transportation of Radioactive Material."
- b. This certificate does not relieve the consignor from compliance with any requirement of the regulations of the U.S. Department of Transportation or other applicable regulatory agencies, including the government of any country through or into which the package will be transported.
- 3. THIS CERTIFICATE IS ISSUED ON THE BASIS OF A SAFETY ANALYSIS REPORT OF THE PACKAGE DESIGN OR APPLICATION
- a. ISSUED TO (Name and Address)

b. TITLE AND IDENTIFICATION OF REPORT OR APPLICATION

QSA Global, Inc. 40 North Avenue Burlington, MA 01803 QSA Global, Inc., consolidated application dated October 9, 2024.

4. CONDITIONS

This certificate is conditional upon fulfilling the requirements of 10 CFR Part 71, as applicable, and the conditions specified below.

5.

(a) Packaging

(1) Model No.: 650L

(2) Description

A welded carbon or stainless steel cylindrical outer shell encases a welded carbon or stainless steel rectangular inner shell. The inner shell contains a titanium "U" tube set in depleted uranium along with internal supports. The tube is crimped in the middle of the "U" to provide a positive stop for the source assembly. Additional shielding is provided by lead or tungsten positioned at various locations around the depleted uranium shield. The Model No. 650L has two source locking assemblies, mounted on the top cover plate, that are used to secure the radioactive special form source, Iridium-192 or Selenium-75, in a shielded position during transport. The packaging measures approximately 10-inches (254 mm) wide, 13.25-inches (337 mm) high and 8.25-inches (210 mm) deep. The maximum weight of the packaging is 90 pounds (41 kg).

(3) Drawings

The packaging is constructed in accordance with QSA Global, Inc., Drawing No. R65006, Revision R, sheets 1-5.

(b) Contents

(1) Type and form of material

Iridium-192 as sealed sources which meet the requirements of special form radioactive material.

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5.(b) Contents (continued)

Selenium-75 as sealed sources which meet the requirements of special form radioactive material.

(2) Maximum quantity of material per package

Ir-192: 240 curies (8.9 TBq) (output)

Se-75: 300 curies (11.1 TBq) (content)

Output curies are determined by measuring the source output at 1 meter and expressing its activity in curies derived from the following: 0.48 R/(h-Ci) Iridium-192 at 1 meter (Ref: American National Standard N432-1980, "Radiological Safety for the Design and Construction of Apparatus for Gamma Radiography").

(3) Maximum weight of contents

0.08 pounds (36 grams), including the mass of radioactive material and the weight of the source capsule handling wire assembly for a shipment containing two source wire assemblies.

(4) Maximum decay heat

Ir-192: 4.8 Watts

Se-75: 1.52 Watts

- 6. The source shall be secured in the shielded position of the packaging by the source assembly. The source assembly must be fabricated of materials capable of resisting a 1475°F fire environment for one-half hour and maintaining its positioning function. The cable of the source assembly must engage the source hold-down assembly. The flexible cable of the source assembly must be of sufficient length and diameter to provide positive positioning of the source at the crimp of the "U" tube.
- 7. The nameplates shall be fabricated of materials capable of resisting the fire test of 10 CFR Part 71 and maintaining their legibility.
- 8. In addition to the requirements of Subpart G of 10 CFR Part 71:
 - (a) The package shall be prepared for shipment in accordance with the Operating Procedures in Chapter 7 of the application, and
 - (b) The packaging shall be maintained in accordance with the Maintenance Program in Chapter 8 of the application.

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- 9. Fabrication of new packagings is not authorized. Fabrication of replacement components needed to support shipment of existing packages is authorized, except for the depleted uranium shield and the inner carbon steel shell.
- 10. The package authorized by this certificate is hereby approved for use under the general license provisions of 10 CFR 71.17.
- 11. Expiration date: November 30, 2030.

REFERENCES

QSA Global, Inc., consolidated application dated October 9, 2024.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

THOMAS BOYCE BOYCE

Date: 2025.02.19 18:18:01 -05'00'

Digitally signed by THOMAS

Tom Boyce, Acting Chief
Storage and Transportation Licensing Branch
Division of Fuel Management
Office of Nuclear Material Safety
and Safeguards

Date: February 19, 2025



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION REPORT

Docket No. 71-9269
Model No. 650L
Certificate of Compliance No. 9269
Revision 14

EVALUATION

By application dated October 9, 2024 (Agencywide Documents Access and Management System Accession No. ML24285A181) QSA Global, Inc. requested renewal of Certificate of Compliance (CoC) No. No. 9269, for the Model No. 650L package. In addition, a consolidated safety analysis report was submitted with this renewal.

Given that no changes have been made to the package itself, the staff continues to find that the package meets the requirements of Title 10 of the *Code of Federal Regulations* (10 CFR) Part 71.

CONDITIONS

In addition to small editorial changes, the following changes have been made to the certificate:

Condition 3(b) was revised to reflect the date of this consolidated application.

Condition No. 11 has been revised to renew the certificate of compliance for an additional five years.

The consolidated application date was added to the REFERENCES section.

CONCLUSION

Based on the statements and representations in the application and the conditions listed above, the staff concludes that the Model No. 650L package design has been adequately described and evaluated, and that these changes do not affect the ability of the package to meet the requirements of 10 CFR Part 71.

Issued with CoC No. 9269. Revision No. 14.